

*Separation of Americium from Neptunium or Plutonium by Back-Extraction.* Experiments show that the Am(III) was quantitatively stripped from the organic phase with 0.05–0.2 M HNO<sub>3</sub>, while most of the Np(IV) and Pu(IV) is remained in the organic phase (Table I). From Table I it can be seen that the separation factor of Am and Np or Am and Pu is more than 50. Consequently, this method may be applied to separating Am from Pu(IV) and Np(IV).

TABLE I. Stripping Data for Am, Np and Pu.<sup>a</sup>

Stripping solution		0.05 M HNO <sub>3</sub>	0.2 M HNO <sub>3</sub>
Back-Extraction Percentage	Am	74	71
	Pu	1.0	1.3
	Np	1.2	1.2
Separation factor	Am/Pu	74	54
	Am/Np	61	59

<sup>a</sup>Organic phase: 30 Vol% DHDECMP-DEB pre-equilibrated with 3 M HNO<sub>3</sub> solution containing <sup>239</sup>Np, <sup>239</sup>Pu and <sup>241</sup>Am tracer. Temp.: 30 °C ± 1 °C. Contact time: 5 min.

## E22

### Investigation of Transport Processes of Actinides by Use of an Analytical Ultracentrifuge for Optimizing the Purex-Process

G. MARX\*, A. BESTANPOURI, C. BINDER, K. H. FELDNER, B. PLEWINSKY and V. FRIEHMELT

*Inst. of Inorg. and Analyt. Chem., Free University of Berlin, Dept. of Radiochemistry, D-1000 Berlin 33, F.R.G.*

Transport data of Actinides in relevant solutions are urgently needed to optimize the electrochemical techniques introduced quite recently into the purex process. Therefore an Analytical Ultracentrifuge has been used for the determination of diffusion coefficients of Th(NO<sub>3</sub>)<sub>4</sub>, Pu(NO<sub>3</sub>)<sub>4</sub>, Np(NO<sub>3</sub>)<sub>4</sub>, NpO<sub>2</sub>-NO<sub>3</sub>, NpO<sub>2</sub>(NO<sub>3</sub>)<sub>2</sub>, PuO<sub>2</sub>(NO<sub>3</sub>)<sub>2</sub> and UO<sub>2</sub>(NO<sub>3</sub>)<sub>2</sub> in aqueous HNO<sub>3</sub> solutions and partially in TBP (30%)-Dodecane-mixtures at 25 °C in the concentration range from 10<sup>-2</sup> M to 10<sup>-1</sup> M, applying Schlieren-, absorptions- and Rayleigh-Interference optics combined with laser techniques. In the Th(NO<sub>3</sub>)<sub>4</sub>-HNO<sub>3</sub>-H<sub>2</sub>O system which stood for the tetravalent actinides, it was shown that the apparent diffusion coefficient ( $D_{app} = 5,8 \times 10^{-6} \text{ cm}^2 \text{ s}^{-1}$ ) equalled the main diffusion coefficient ( $D_{main} = 6,3 \times 10^{-6} \text{ cm}^2 \text{ s}^{-1}$ ), proving that the cross term could be neglected. An analytical ultracentrifuge can also be used for investigating and determining the kinetics and distributions of extraction processes.

Moreover diffusion coefficients of UO<sub>2</sub>(NO<sub>3</sub>)<sub>2</sub>·2TBP in pure dodecane, pure TBP and TBP-dodecane mixtures (30%/70%) were determined within the concentration range from 5 × 10<sup>-3</sup> M to 5 × 10<sup>-2</sup> M. The obtained results plotted against concentration give straight lines.

In pure dodecane the diffusion coefficient of UO<sub>2</sub>(NO<sub>3</sub>)<sub>2</sub>·2TBP was found to be 3 × 10<sup>-6</sup> cm<sup>2</sup> s<sup>-1</sup>, this value decreasing for 10% by adding 30% TBP.

The diffusion coefficients were compared with electrochemical transport data, obtained earlier from conductance and transference measurements, also with limiting ionic conductances of Am<sup>3+</sup> and Cm<sup>3+</sup>, found quite recently.

## E23

### Lanthanide Orthophosphate Ceramics for the Disposal of Actinide-Contaminated Nuclear Wastes<sup>†</sup>

L. A. BOATNER\*, M. M. ABRAHAM and B. C. SALES

*Solid State Division, Oak Ridge National Laboratory, Oak Ridge, Tenn. 37830, U.S.A.*

Since the half lives of some of the hazardous, α-active actinide isotopes contained in nuclear wastes are relatively long, severe stability requirements are imposed on the materials that must function as primary containment media in a permanent actinide waste-disposal system. Such materials must maintain a high degree of physical and chemical integrity under a variety of potentially hostile geological environments, including possible hydrothermal conditions. Investigations by McCarthy *et al.* [1] of the stability of borosilicate glasses under hydrothermal conditions led to reservations regarding their suitability as primary hosts for the disposal of nuclear wastes during the desired 10<sup>4</sup> to 10<sup>5</sup> year immobilization period. These concerns, in turn, resulted in renewed interest in alternative materials for the disposal of radioactive wastes.

One approach employed in selecting alternatives to borosilicate glass consisted of a process of examining various mineral systems whose long-term stability was well established geologically. An approach of this type led to the SYNROC concept of Ringwood [2] and also to the decision to investigate the lanthanide orthophosphates that are analogs of the uranium and thorium bearing mineral monazite [3] (La, Ce, ...PO<sub>4</sub>). Geological evidence indicates that monazites have survived various rock-forming and

<sup>†</sup>Research sponsored by the Division of Materials Sciences, U.S. Department of Energy under contract W-7405-eng-26 with the Union Carbide Corporation.